

CLOSING SESSION 6 (Summary Presentation)

Chairperson: M. Bakirov

In the frames of Section 6 «Safety Assessment in NPPs» there were presented 6 papers from the following countries: Russian Federation, Czech Republic, India, Spain, Ukraine and paper from IAEA.

At present time increasing requirement to reliability and safety of the equipment of NPP. In connection with it safety assessment in NPP is on of the main task in operation of water cooled reactor in 21st Century.

Very interesting philosophy presentation was made by Dr. M. El-Shanawany from IAEA. There was presented the International point of view to this problem. Such presentation gave very fruitful beginning of the Section.

In the paper from Russia «Development of Universal Methodology of Specimen Free Nondestructive Inspection of Mechanical Properties of NPP Equipment Metal in All Stages of Lifetime», taking into account of 25 years operation experience in direction of use on NPP of methods and means of inspection of mechanical properties, there were presented the following:

- Development of algorithm of drawing up of International program of ND control for various types of equipment.
- Development of strategy on the base of ND methodic of procedure of certification universal methodic control of mechanical properties of NPP equipment metal in all stages of lifetime.

Presentation from Czech Republic was made by Mr. Jozef Misak «Comparison Between International and Slovak Design Safety Requirements...»

The presentation consist of two parts. In the first part it presents comparison between IAEA Safety Standards, VENRA Reference Levels and Slovak National Standards. From the comparison it is shown that there is good consistency between VENRA Reference Level, IAEA Safety Requirements and Slovak Legislative Documents.

The next Paper was presented by Mr. Krishna Kumar from India. The title of presentation is “Safety Assessment and Improvement in Indian NPPs”. There are presented the following directions:

- Life extension and safety assessment of BWR TAPS 1,2.
- Retrofitting of components and upgrading for the old PHWR units.
- Improvement for facility system maintenance.
- Accident management.

One of the most important question is the Human Factor during the accident progression.

In presentation from Spain, which was made by Mr. Riverola, “Experience Feedback of Current LWR on the Design and Operation of Advanced LWR, under the Safety Analyses Point of View” is accumulated the experience in Generation II reactors in the direction of safety analysis.

It can be concluded, that Safety Analysis as a knowledge framework will keep being important in supporting operation not only in life extended current reactors, but also of advanced LWR.